

Production and Isolation of ^{237}U and Fission Fragments Resulting from the $^{238}\text{U}(\gamma, n)$ and $^{238}\text{U}(\gamma, f)$ Reactions

August 1, 2017

**13th International Topical Meeting on
Nuclear Applications of Accelerators**

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INL/CON-17-41496

Objective

Photonuclear Production

- Predictive modeling
- Irradiation techniques
- Identification techniques
- Isotope purification
 - Chemical processes
 - Mass separation

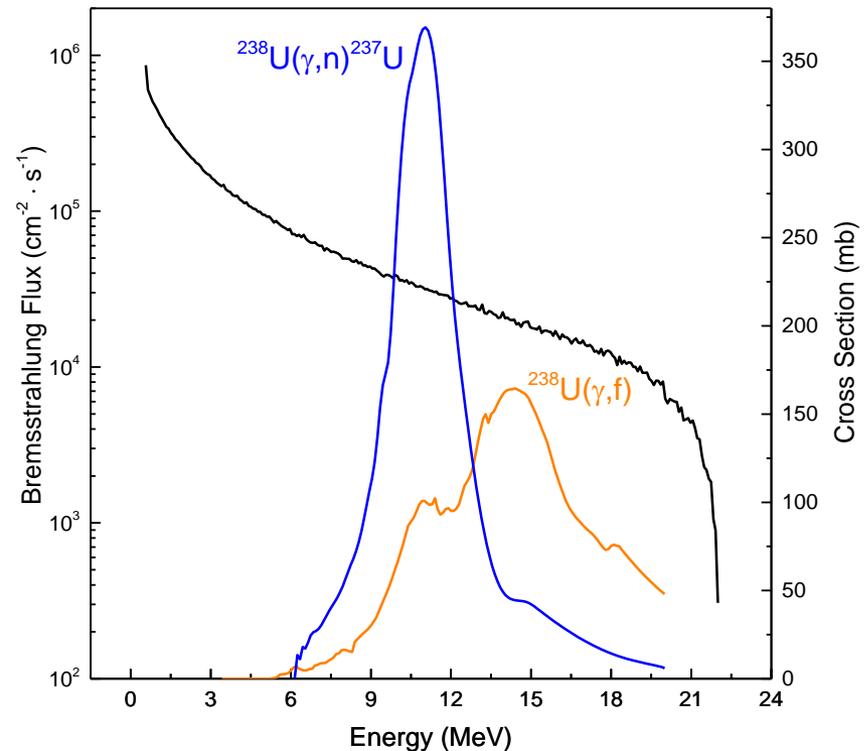
Demonstrate ^{237}U and fission fragment production from $^{238}\text{U}(\gamma,n)$ and $^{238}\text{U}(\gamma,f)$ reactions.

Radioisotope Uses

- Tracers
- Nuclear forensics
- Medical isotopes

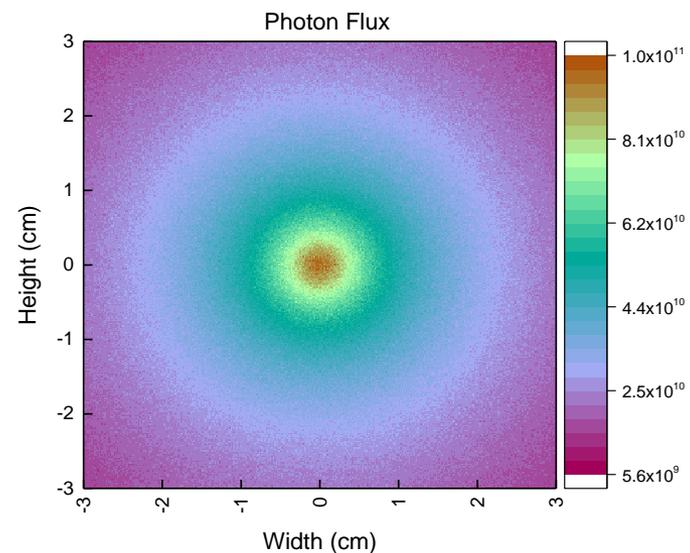
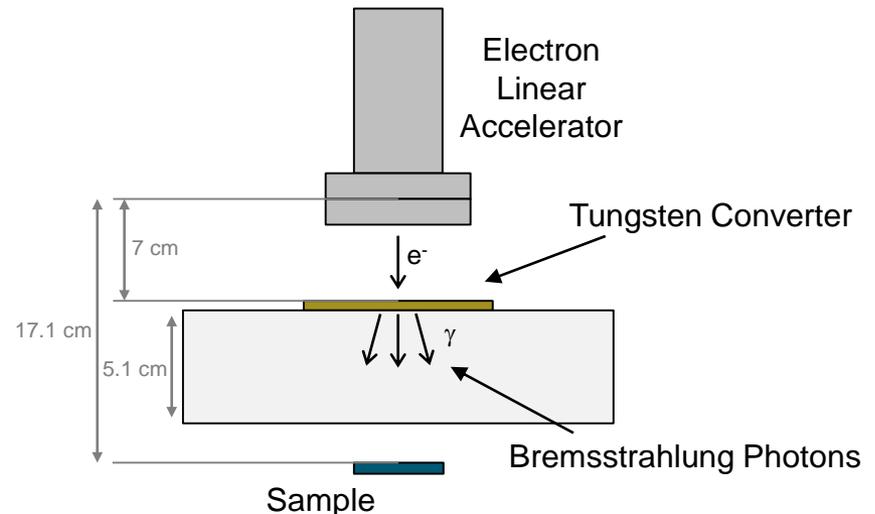
Photonuclear Production

- Idaho Accelerator Center electron linear accelerator → tungsten converter → bremsstrahlung
- Bremsstrahlung curve simulated with MCNP6
- Cross sections obtained from the Evaluated Nuclear Data File
 - At the lower end of energies the production of ^{237}U is more likely
- Accelerator energy increase:
 - greater flux magnitude
 - higher photon endpoint energy
 - photons more forward directed



Predictive Modeling Using MCNP

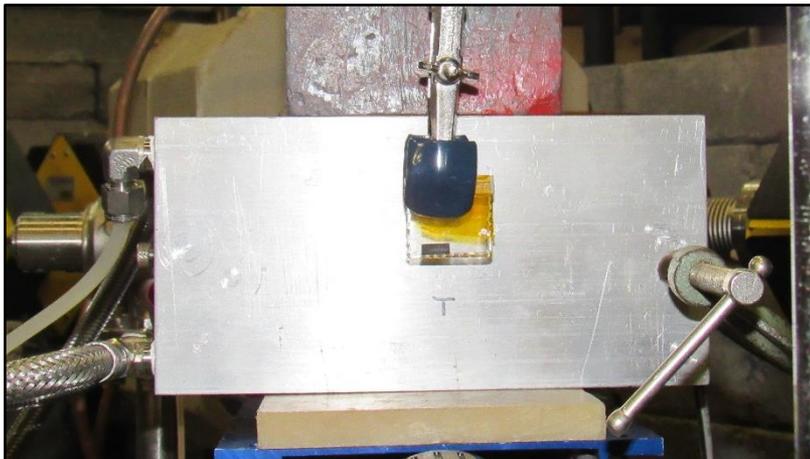
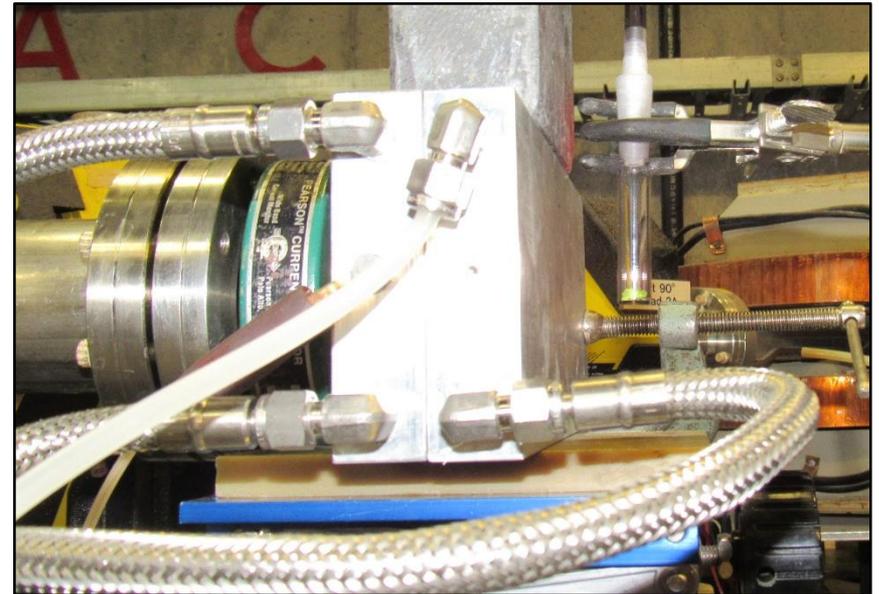
- Help determine optimal experiment geometries
- Uranium foil fully positioned within the beam diameter 10 cm from the converter
- Predict sample activity
 - Transport
 - Radiation safety
 - Production Yield
 - (ENDF/B-VII.1 (γ, n) cross section)
 - 1 hr. irradiation
 - 21.5 MeV
 - 0.1 g
 - $\sim 90 \mu\text{Ci} \cdot \text{kW}^{-1} \cdot \text{hr}^{-1} \cdot \text{g}^{-1}$



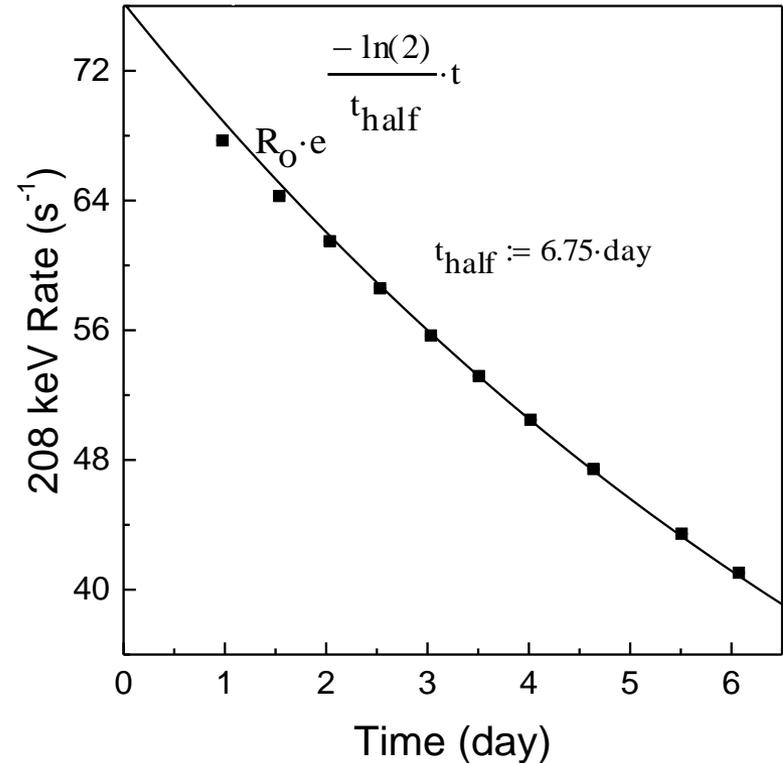
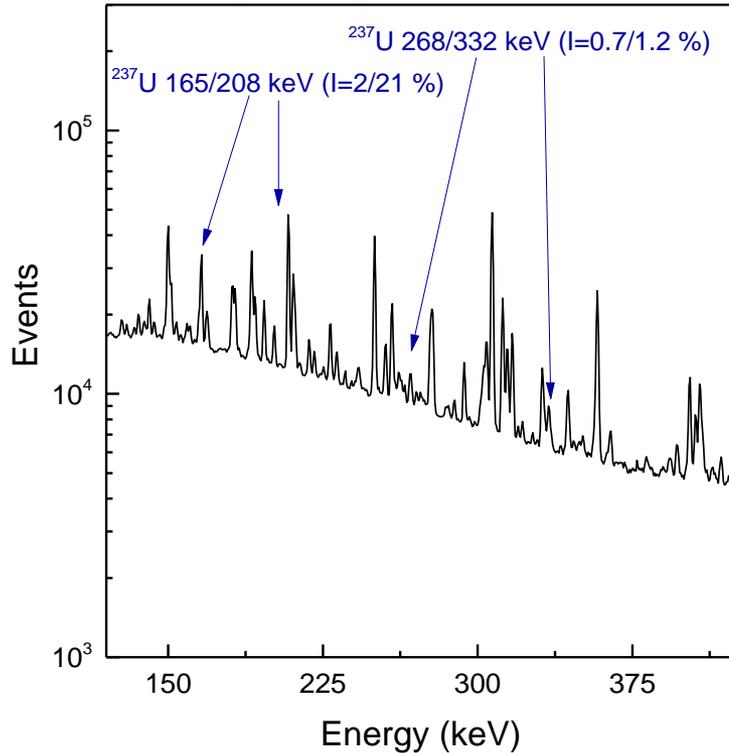
- $\sim 50\%$ within (-1,1)
- $\sim 70\%$ within (-2,2)
- $\sim 85\%$ within (-3,3)

^{238}U Sample Irradiation

- Optimal accelerator energy
 - Stable 21.5 MeV
 - Endpoint energy beyond peak of cross section.
- Optimal accelerator current
 - Largest achievable
- Experiment parameters
 - 50 μA
 - 0.1 g $^{\text{nat}}\text{U}$ foil



Identifying ^{237}U



Detection Details

- 4 hrs. post irradiation
- 15 hr. detection

Cross Section Details

- $^{238}\text{U}(\gamma, n)$ Threshold: ~ 6.2 MeV
- Competes with $^{238}\text{U}(\gamma, f)$

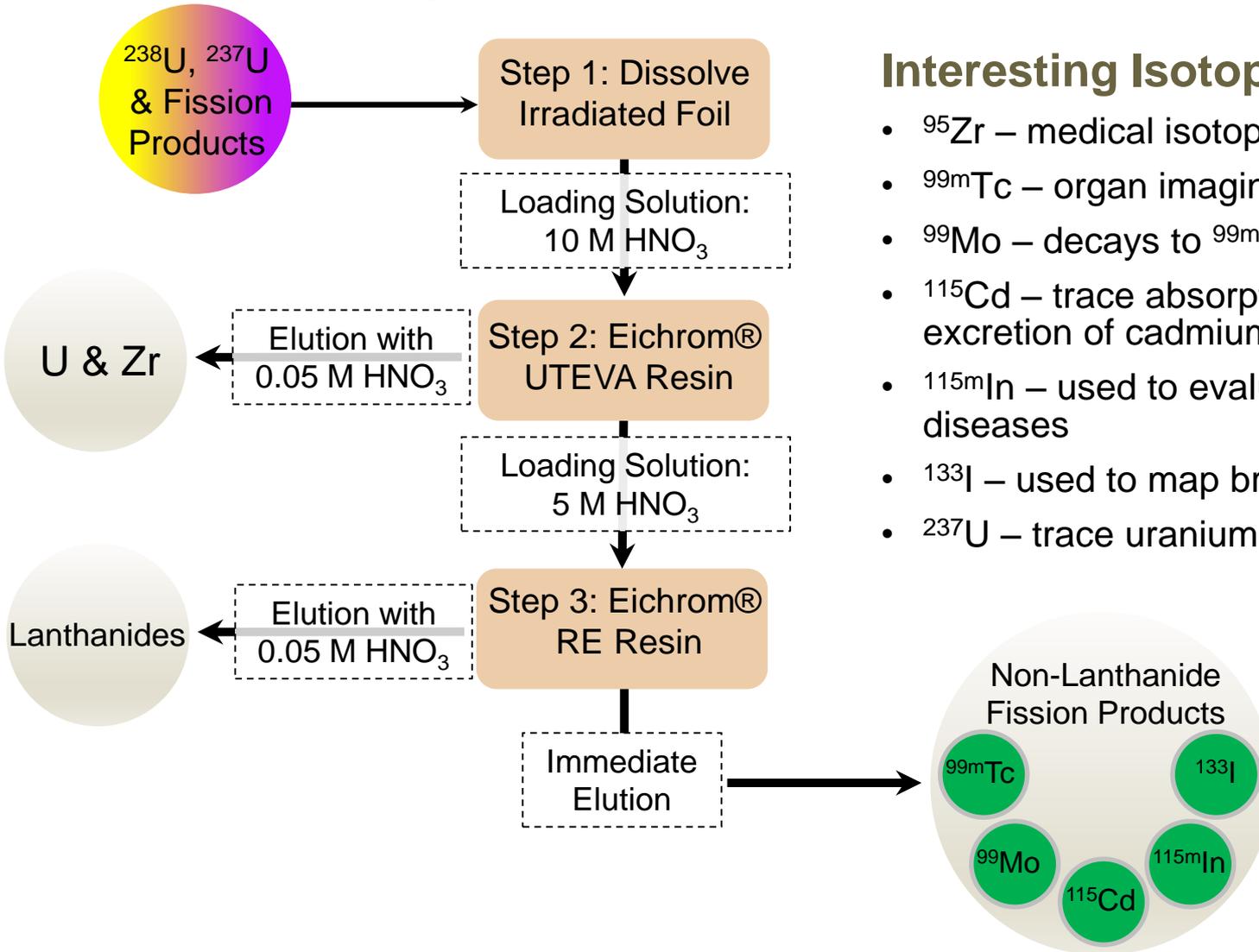
Identification

- Observed 4 photo-peaks (FP interferences)
- Data trends to half-life curve

Production Yield

- $\sim 98 \pm 0.1 \mu\text{Ci} \cdot \text{kW}^{-1} \cdot \text{hr}^{-1} \cdot \text{g}^{-1}$ (9% difference)

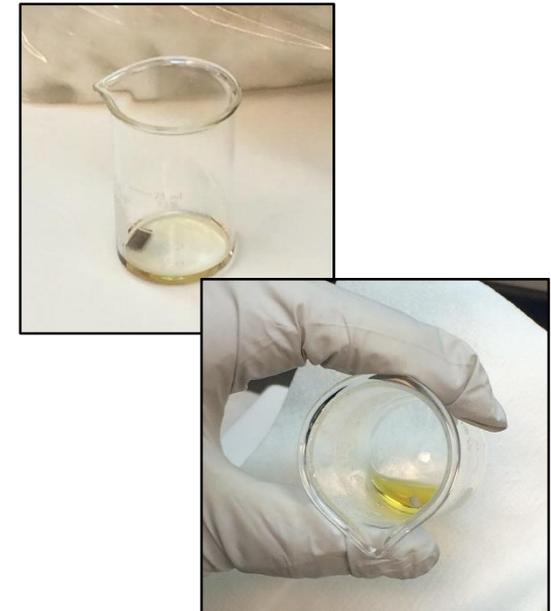
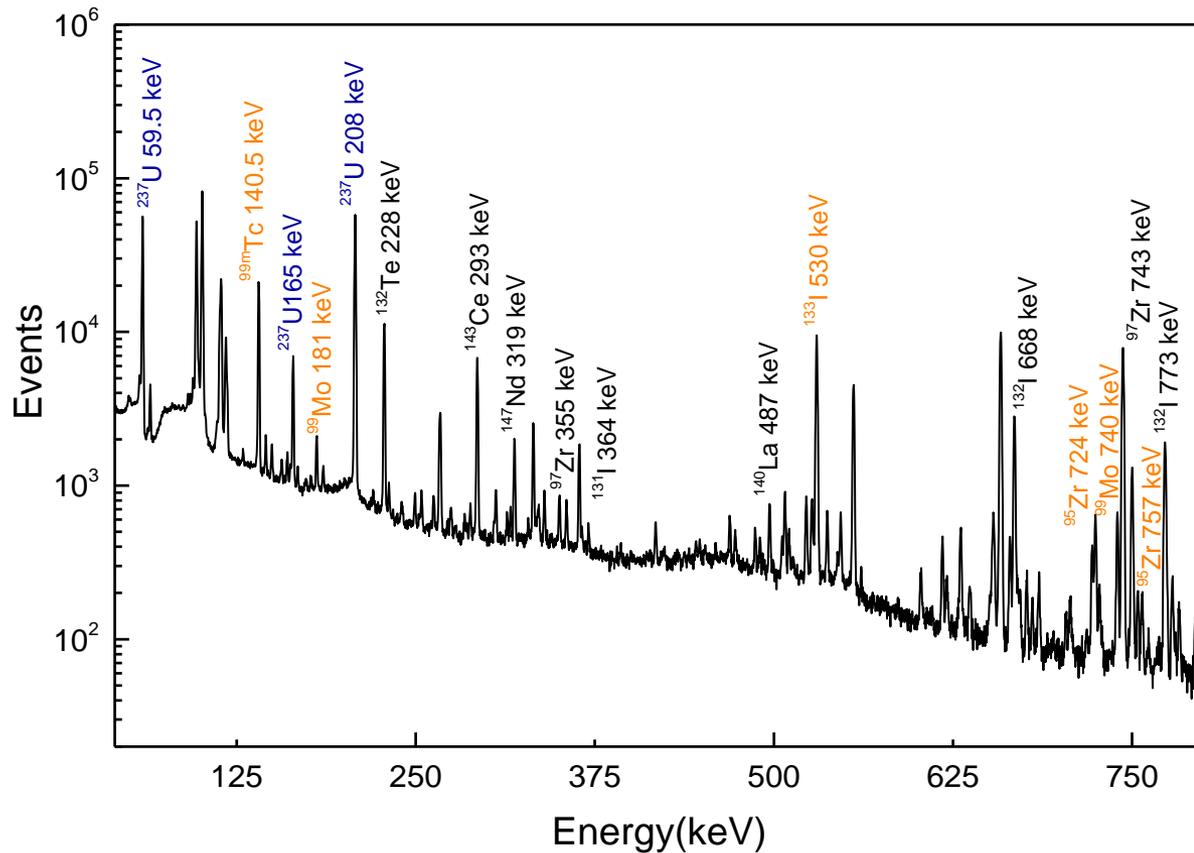
Chemical Separation at INL



Interesting Isotopes

- ^{95}Zr – medical isotope
- $^{99\text{m}}\text{Tc}$ – organ imaging
- ^{99}Mo – decays to $^{99\text{m}}\text{Tc}$
- ^{115}Cd – trace absorption and excretion of cadmium in tissues
- $^{115\text{m}}\text{In}$ – used to evaluate certain diseases
- ^{133}I – used to map brain tumors
- ^{237}U – trace uranium in ground water

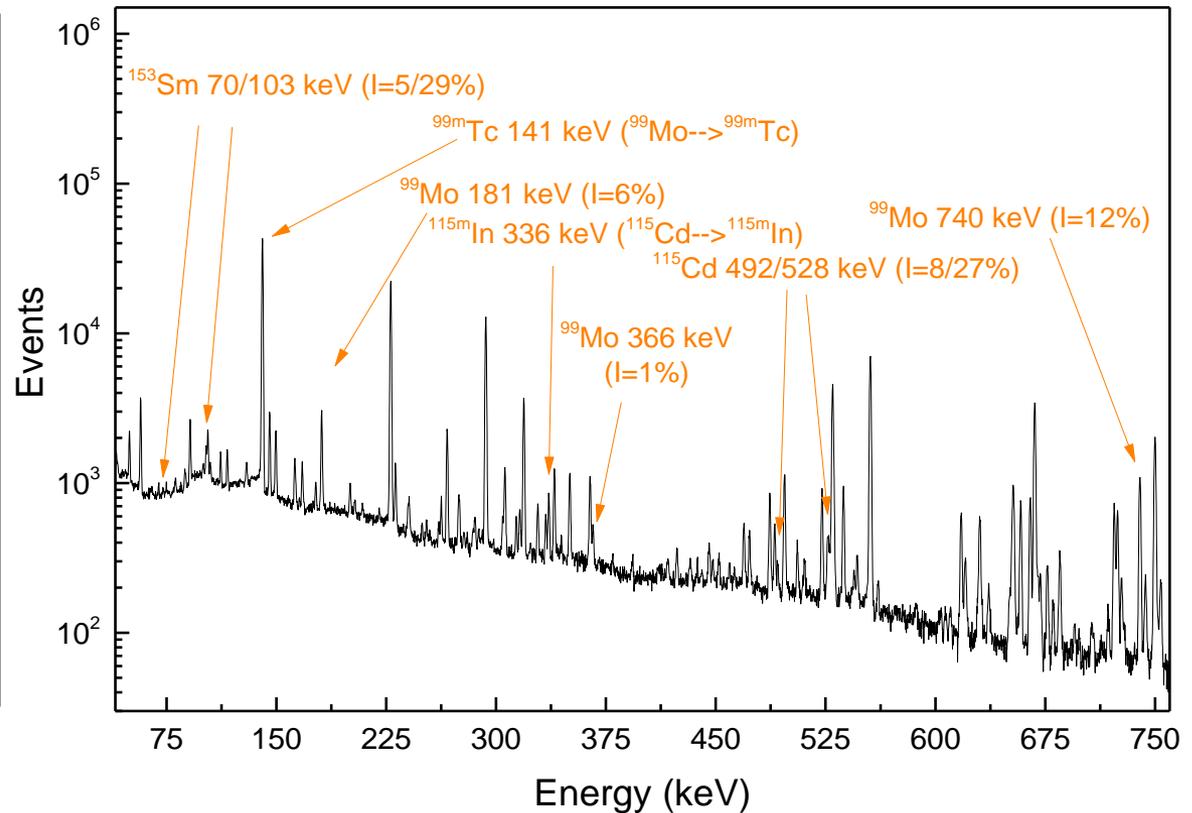
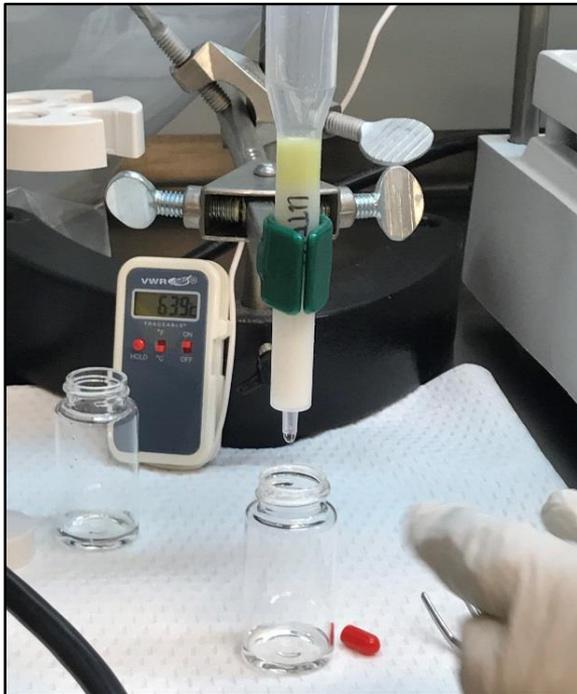
Foil Dissolution



Dissolution Details

- ~1 hr.
- 10 M nitric acid

UTEVA Column



Half-lives

- ^{99}Mo – 2.8 days
- $^{99\text{m}}\text{Tc}$ – 6.0 hrs
- ^{115}Cd – 2.2 days
- $^{115\text{m}}\text{In}$ – 4.5 hrs
- ^{153}Sm – 1.9 days
(cancer treatment/diagnostics)

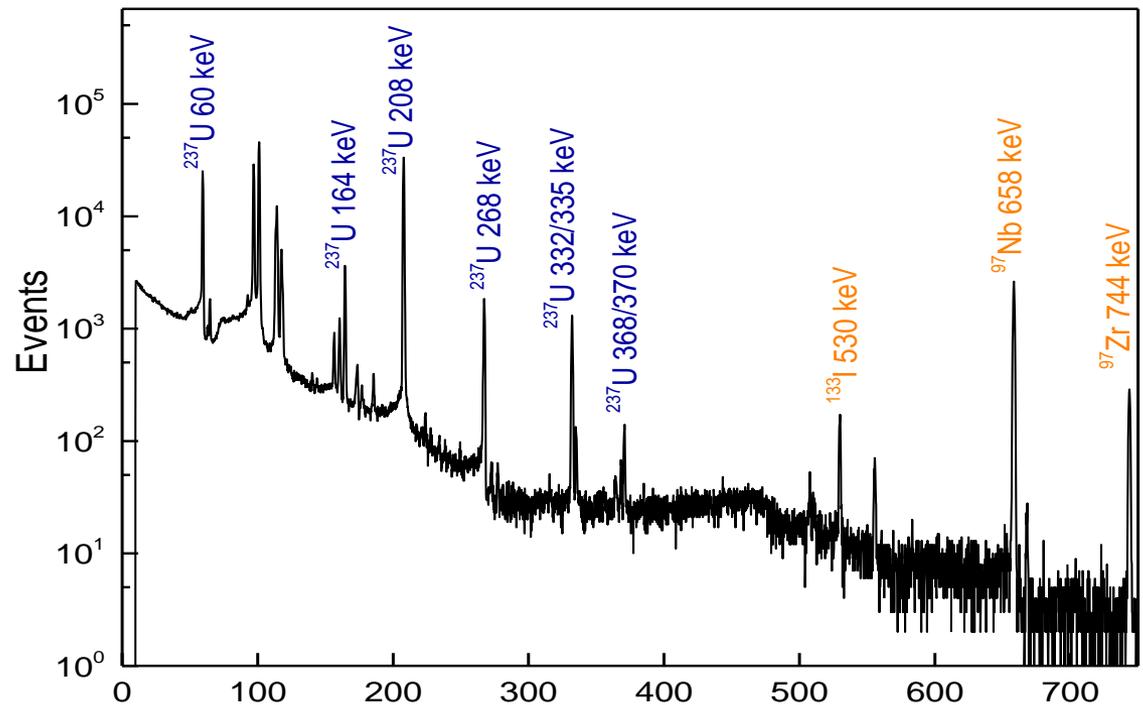
Detection Details

- UTEVA rinse
- 27.5 hrs. post irradiation
- 0.7 hr. detection

Elution from UTEVA Resin

Half-lives

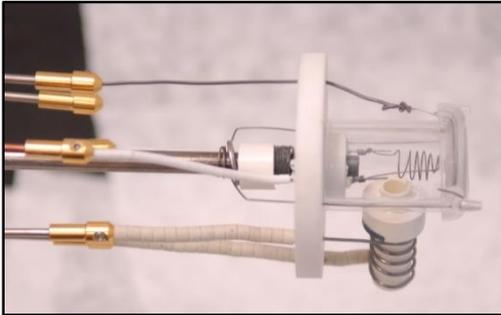
- ^{237}U – 6.8 days
- ^{97}Zr – 16.7 hours
- ^{237}U most abundant radioisotope: ~ 6 days
- 87% of fission products with half-life < 1 day
 - 8% > 6 days



Detection Details

- Elution
- 26.3 hrs. post irradiation
- 30 minute detection

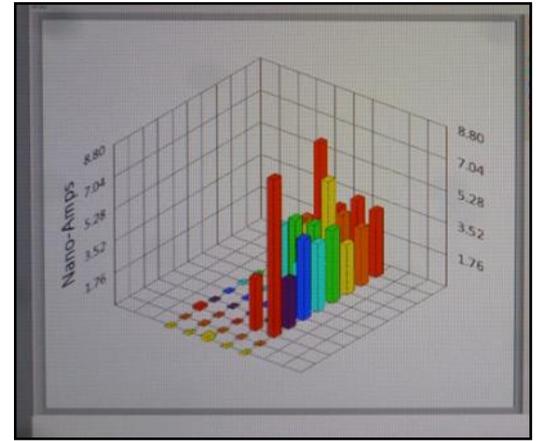
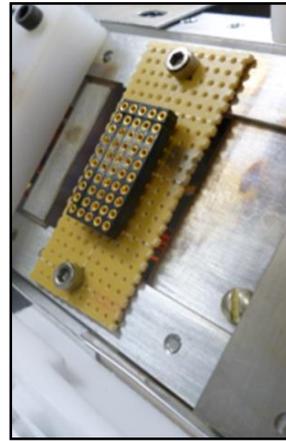
Isotopic Purification



Mass separation

- Isotopes separated with mass separator
- The uranium sample is positively ionized thermally and by electron current
- Ion beam propagates through a bending magnet
- Lighter isotopes are deflected more sharply

Isotopic Purification



Catcher Foils

- Ion beams propagate into catcher foils
- 1% collection efficiency
- 5×10 Faraday cup catches charged particles used to determine the number of ions
- Pixel array readout characterizing the beam profile

Conclusions & Future Agenda

- Demonstrated photonuclear production of:
 - ^{237}U , $^{99\text{m}}\text{Tc}$, ^{99}Mo , ^{115}Cd , $^{115\text{m}}\text{In}$, ^{133}I , ^{153}Sm
- Optimize production and increase activities
 - Modify accelerator parameters
- Optimize sample transport
 - Goal under 3.5 hrs. for packaging and transport
- Optimize separation for isotopes of interest
 - Separation goals:
 - Uranium \rightarrow 2.5 hrs.
 - Non-lanthanides in 3 hrs.
 - Strategy to optimize for each isotope of interest
 - Lanthanides < 7.5 hrs.
- Continue mass separator development
 - Optimize uranium separation
 - Introduce new radioisotopes



Thank you.